



Radiological characterization of high-energy proton targets from the CERN-ISOLDE facility

M. Rababah, R. Catherall, G. Dumont, M. Magstris, P. Pisano, S. Stegemann, C. Theis, J. Vollaire

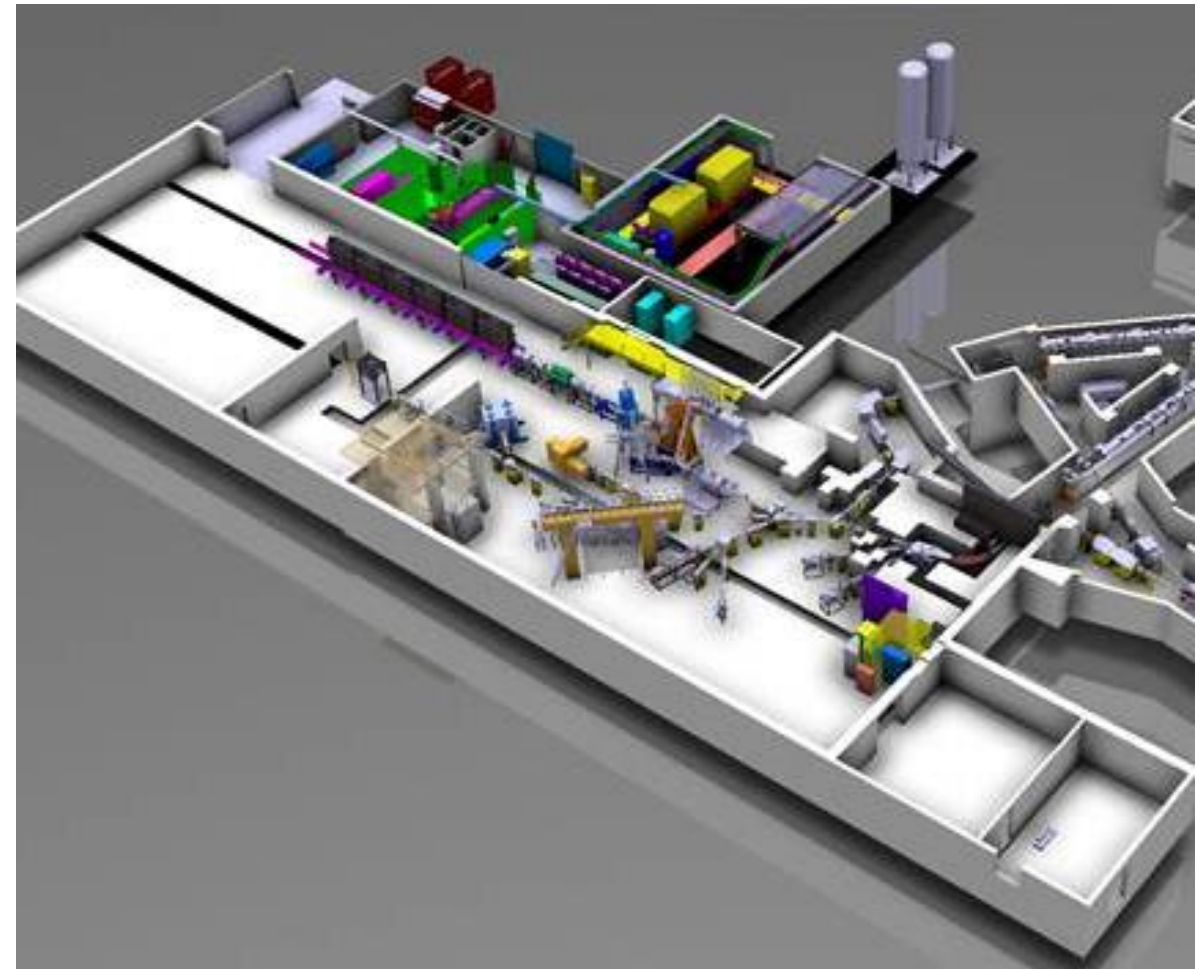
21/09/2022

Contents

- **Introduction to ISOLDE facility**
- **Waste produced at ISOLDE**
- **Challenges**
- **Estimation methodology**
- **General considerations**
- **Conclusions**

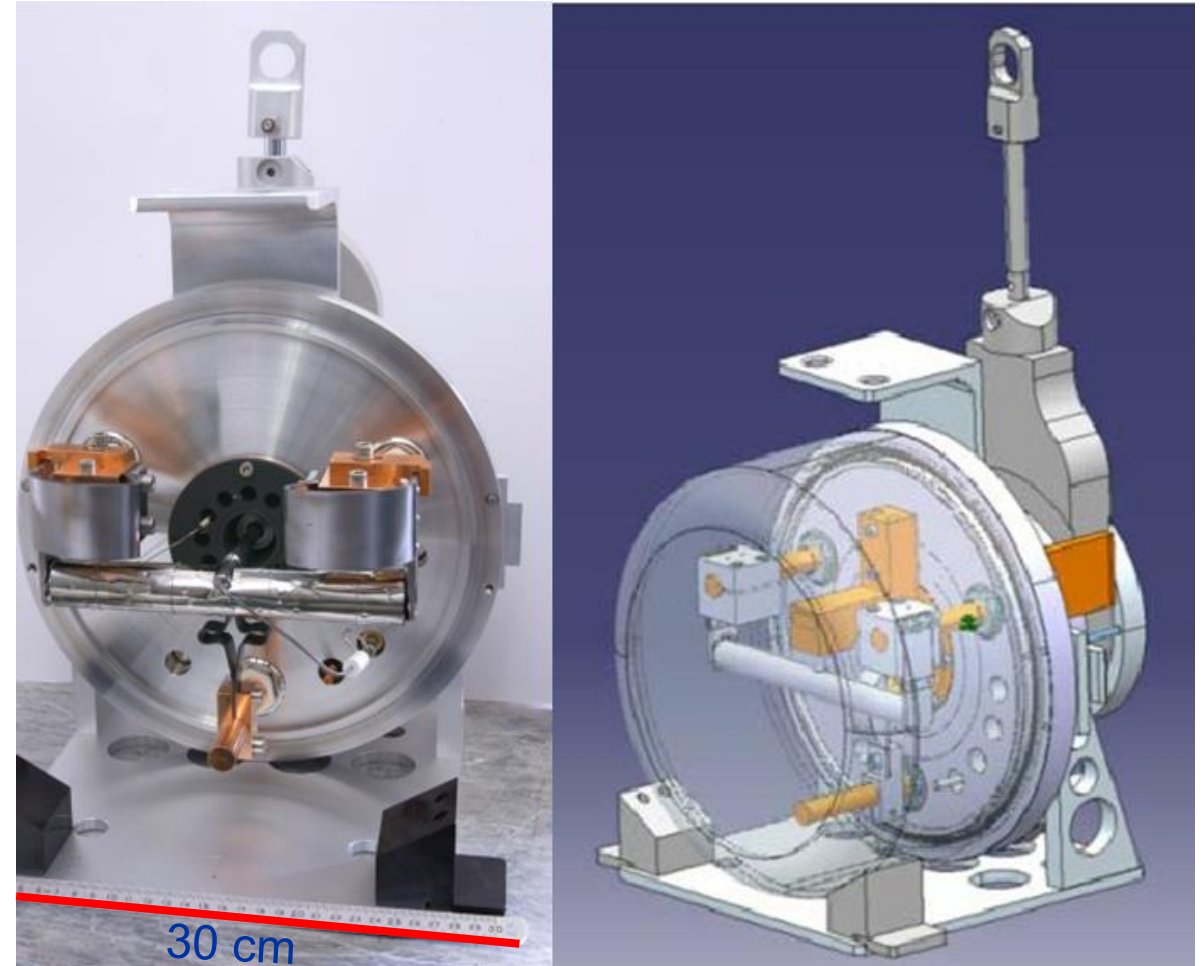
ISOLDE Facility

- The ISOLDE facility (Isotope mass Separator On-Line DEvice) at CERN is a unique source of beams of radioactive nuclides.
- It is used in a wide range of research domains.
- The radioactive nuclides are produced via spallation, fission and fragmentation reactions originating from a 1.4 GeV proton-beam that impinges on specifically designed thick targets.



ISOLDE Targets

- Target is a complex assembly that comprises the target core and its container, the ion source, the target base and the auxiliary components.
- 40 different targets (targets material , ion sources and configuration) are generated irradiated every year.
- Different target configurations are required for the different physics experiments.
- Typical irradiation times are from few days to a couple of weeks.



ISOLDE- Waste Production

- After their irradiation, targets are stored for radioactive decay.
- Hundreds of irradiated targets are currently stored at CERN and a similar number is expected to be irradiated over the period 2022-2031.
- These targets will have to be dismantled and disposed of as radioactive waste in Switzerland.
- Therefore, the radiological characterization is a crucial step for their successful disposal.

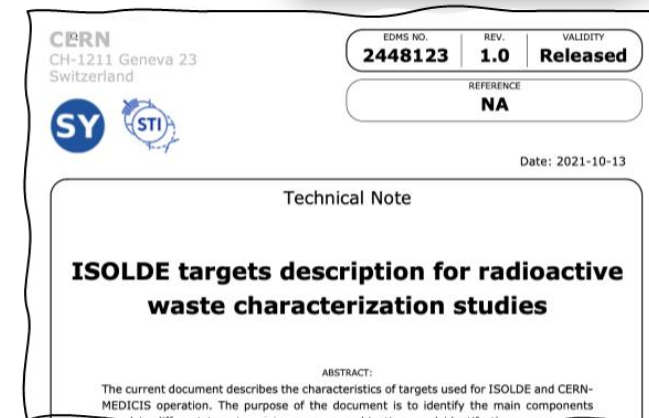
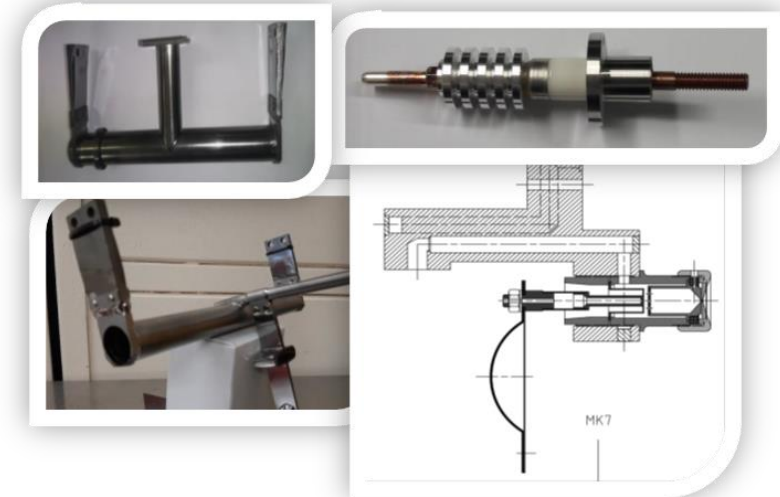


Challenges

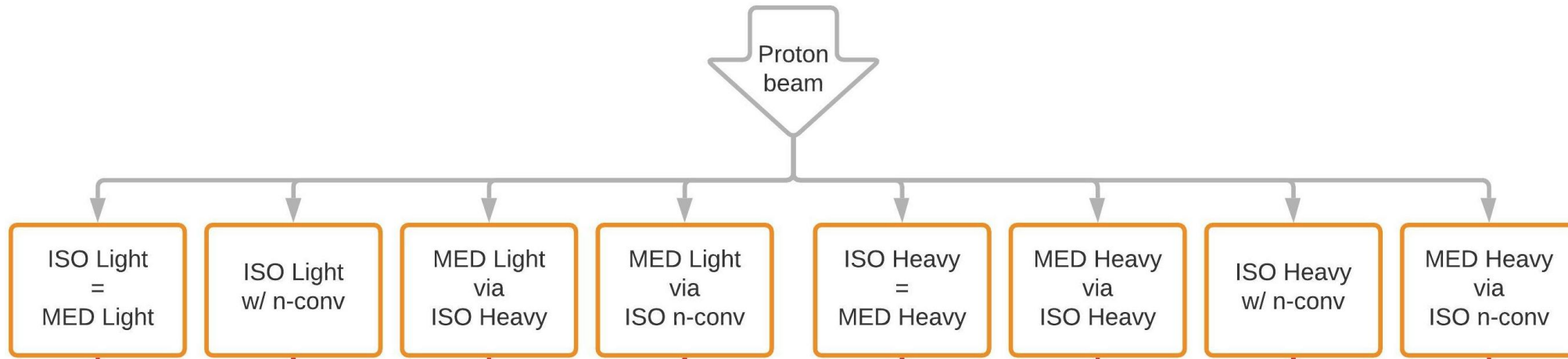
- **There are many challenges associated with characterizing such radioactive waste**
- **These are mainly due to:**
 - Different target designs and characteristics (e.g. core materials)
 - Different irradiation configurations (e.g. direct/indirect irradiation)
 - Different irradiation histories (e.g. cooling times)
- **What did we do?**
 - Identifying the parameters relevant for the radiological characterisation
 - Calculating the fluence spectra using Fluka
 - Performing activation calculations using ActiWiz
 - Performing extensive statistical analysis of the results

Challenges - Target characteristics

- 5 categories were identified to cover all the possible target designs.
- For each category, we calculated the total mass of each structural material (12 structural materials in total).
- For each of the 17 core materials, we retrieved the total mass and composition.
- For each material, either core or structural, we computed the average mass and standard deviation.



Challenges - Irradiation configurations



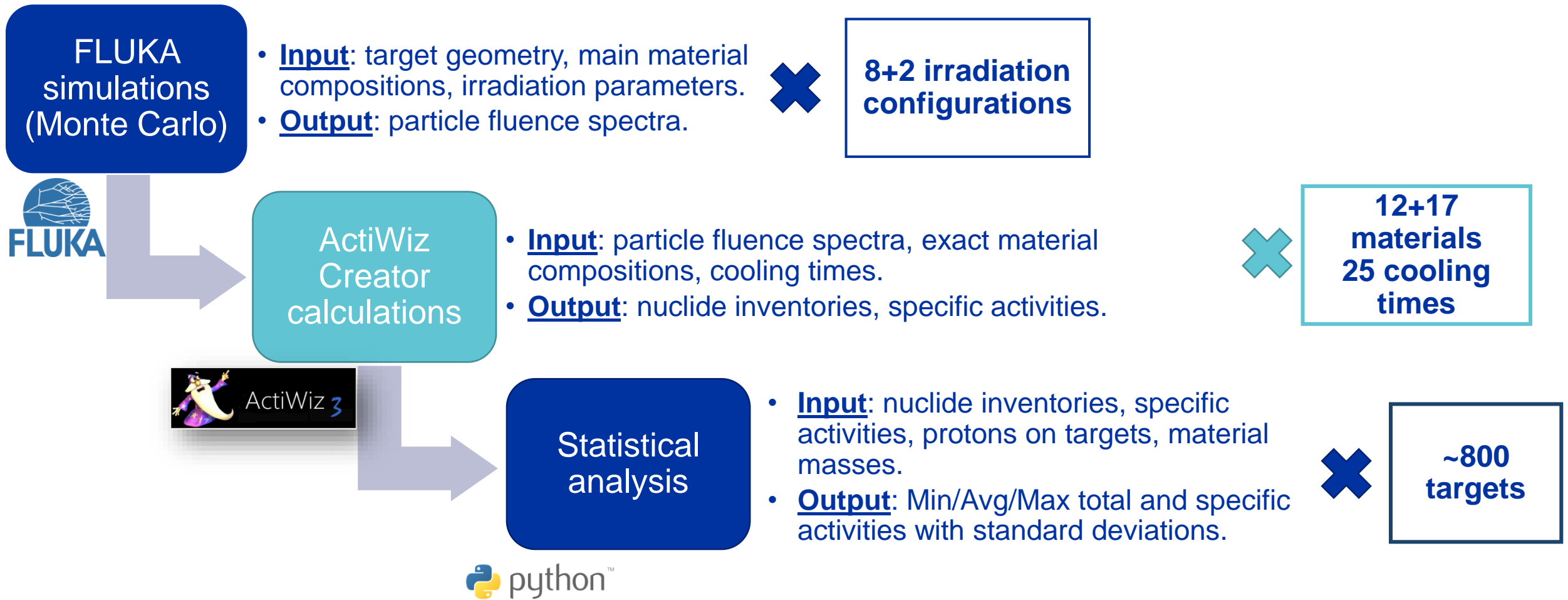
8 irradiation configurations to simulate the activation of the structural materials using reference core materials:

- **Light: Tantalum**
- **Heavy: UC2C**

2 irradiation configurations to simulate the activation of the core materials:

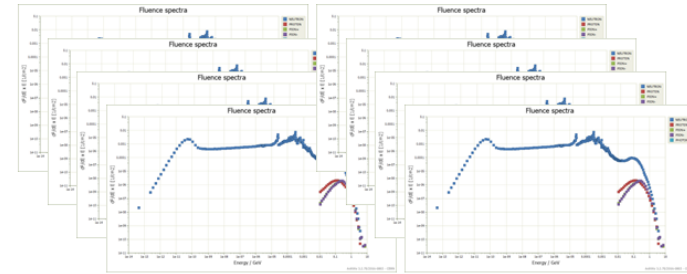
- **Direct irradiation of each core material**
- **Irradiation via n-converter of each core material**

Estimation methodology – Calculation scheme



Estimation methodology – Example

- Each ISOLDE-MEDICIS target contains 13.4 kg of Aluminium.
- It can be exposed to slightly 8 different particle spectra - 8 different irradiation configurations.
- We compute the nuclide inventories produced by each of the 8 different particle spectra for each cooling time.
- We compute the average nuclide inventory (given the previous 8) for each cooling time.
- We compute the average nuclide inventory for each target considering the target-specific number of received protons and the end-of-irradiation date.



$$SA_i(mat, t, irr_c), \text{ for } i \in [1, N], c \in [1; C]$$

Representative nuclide inventory for Al, for a given cooling time, per received proton.

$$\overline{TA}_i(mat) = \sum_{k=1}^K \bar{A}_i(mat, t_k) \cdot POT_k, \text{ for } i \in [1, N]$$

General considerations

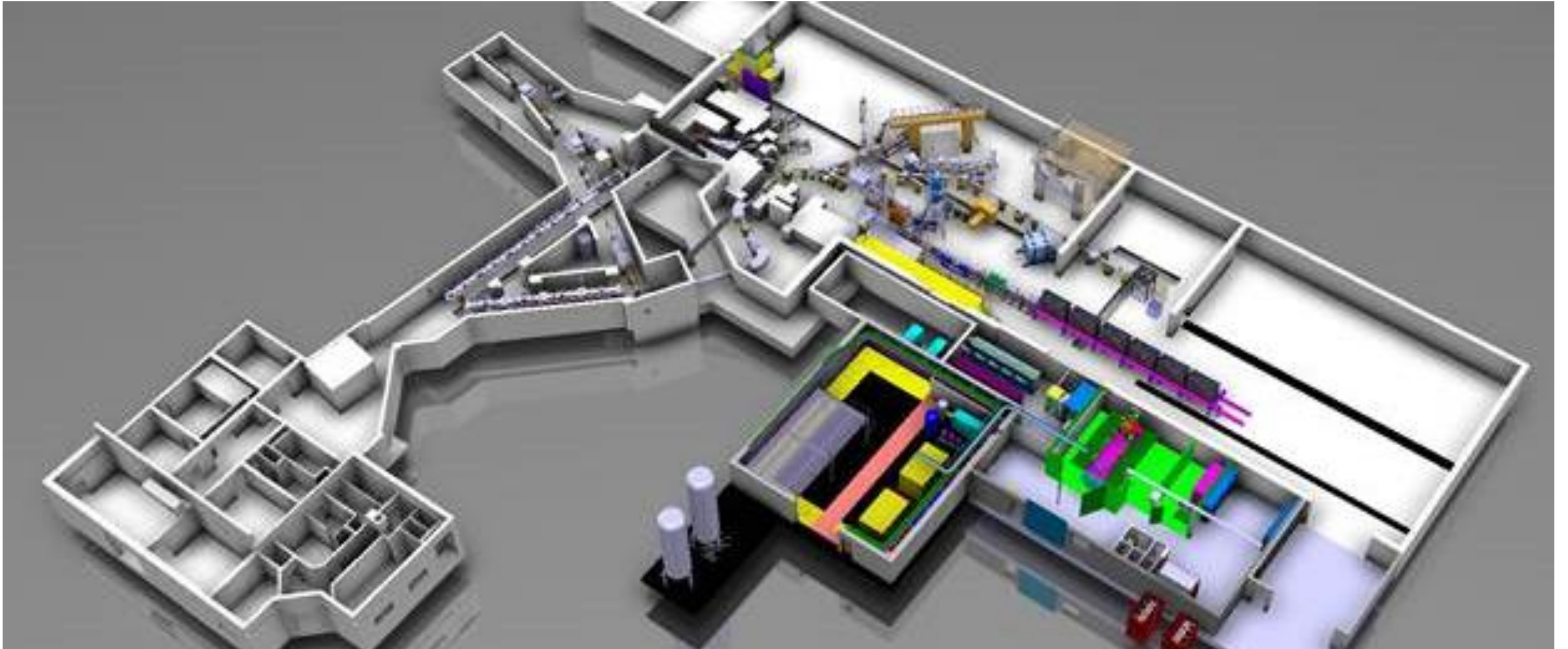
- It is preferable to create a database with the extended ActiWiz results.
- JSON format provides an excellent, easy-to-read data structure for storing and processing the results.
- The combined use of FLUKA, ActiWiz Creator and Python (or C++, R, etc.) for the statistical analysis allows:
 - Benefitting from each tool's capabilities – modelling options, computational speed, analysis versatility.
 - Using each step's output for other studies (e.g. using the FLUKA spectra for different AW calculations).

```
"3y": {  
  "H-3": {  
    "Name": "H-3",  
    "AvgSpecAct": 4.467415250139564e-16,  
    "RelStDevSpecAct": 0.2473454850745016,  
    "MaxSpecAct": 6699.483587496967,  
    "AvgTotAct": 5.973380930961611e-12,  
    "RelStDevTotAct": 0.24734690463141132,  
    "AvgLLFraction": 4.467415250139564e-18,  
    "HalfLife": "389096898.56599998"  
  },  
  "C-14": {  
    "Name": "C-14",
```

Conclusions

- **The structural and core materials are studied separately. 5 categories and 17 core materials were defined to cover the possible target designs.**
- **The estimation methodology**
 - Is comprised of 3 steps - FLUKA, ActiWiz, statistical analysis (Python),
 - Is designed to cover all the possible combinations of target designs, core materials, irradiation configurations, and cooling times.
 - Results in average nuclide inventories (total and specific activities) for each material considering the target-specific cooling time and number of primary protons.
 - Computes the standard deviation by considering the activity spread (produced by the irradiation configurations) and the mass spread.
- **ActiWiz was used to create a database rather than only the specific results required by the study.**
- **By combining Monte-Carlo (FLUKA) and analytical calculation (ActiWiz), and statistical analysis, one can manage to predict activities and scaling factors for extremely heterogeneous waste.**

Questions





home.cern