



# Radionuclide inventory of burnable radioactive waste contaminated with artificial alpha-emitters

M. Rababah, K. Bauer, G. Dumont, M. Magistris, N. Mena, P. Pisano, C. Theis

21/9/2022

# Contents

- **Radioactive waste at CERN**
- **Radioactive burnable waste**
- **Radiological characterization**
- **BEEP-ABG Characterization**
- **Conclusion**

# Radioactive waste at CERN

CERN's RW Classification		Elimination pathways
<b>Clearance candidates - CL</b> (Candidats à la Libération inconditionnelle)	Release from regulatory control in <b>Switzerland</b> (Clearance <> “free-release”)	AMAL, EVA, B-FREE, PLATAN, CRANES, etc.
<b>Very Low-Level Waste - TFA</b> (Très Faibles Activités)	Surface disposal in <b>France</b> . As defined by the acceptance criteria of the ANDRA CIRES repository	BEEP-ABG, BEEP-BG, ELICA, ELMA, SHERPA, ELFI, etc.
<b>Intermediate &amp; Low-Level Waste - FMA-VC</b> (Faibles et Moyennes Activités à Vies Courtes)	Surface disposal in <b>France</b> Half life $T_{1/2} < 30y$ As defined by acceptance criteria in ANDRA CSA repository	ABEILLE, MAST, etc.
<b>Intermediate &amp; Low-Level Waste - FA-MA</b> (Faibles Activités et Moyennes Activités)	Disposal in <b>Switzerland</b> When FMA-VC acceptance criteria (half-life, activity level) are not met	STEP-1, ITEP, STEP-2, etc.

**93% in volume of the waste stored at CERN belongs to the category of TFA**

# Burnable radioactive waste

## Type of waste

- Mostly personal protection equipment
  - Gloves, fabrics, Over-shoes, etc.
- ventilation filters.

## BEEP-Free

- Elimination with free release in Switzerland

## BEEP-BG

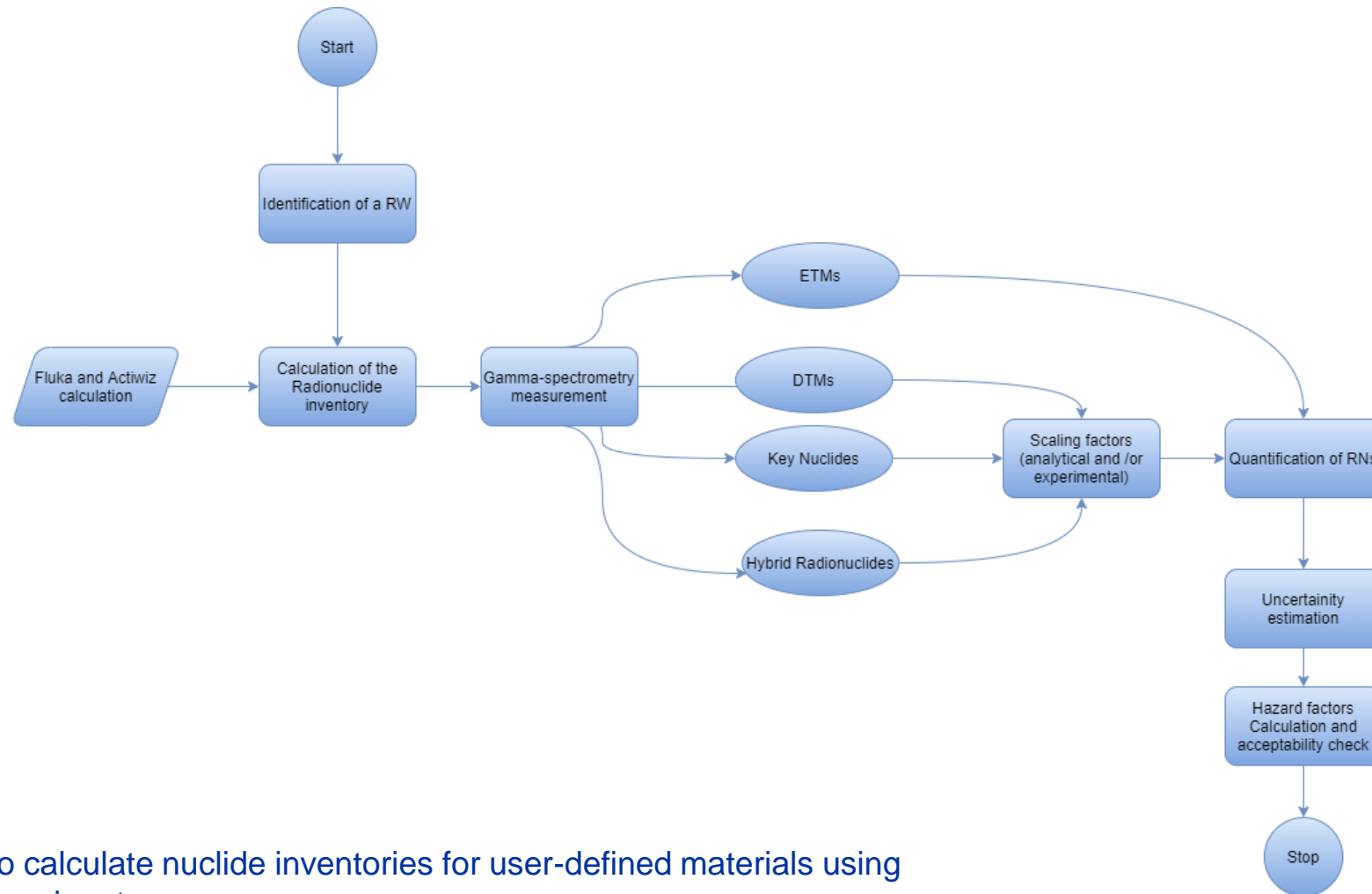
- Contamination by activated metallic particulates or concrete dust

## BEEP-ABG

- Contamination in the manufacturing and handling of spallation targets (n\_TOF) and ISOLDE



# Characterization of TFA



Activiz creator allows to calculate nuclide inventories for user-defined materials using particle fluence spectra as input

Courtesy of B. Zaffora, M. Magistris

# Challenges for predicting BEEP-ABG RN inventory

- **Wide range of activation scenarios**
  - Targets of ISOLDE and n\_TOF facilities – target core and structural materials
- **A sampling strategy is not adequate given the heterogeneity of the waste**
  - Hence relying only on FLUKA simulation and analytical calculation
- **A longer list of DTMs**
  - Targets made of actinides produces virtually any radionuclide from nuclides table.
  - Alpha emitters DTMs: Gd-148, Po-210, etc.

# ISOLDE and n\_TOF

- **n\_TOF targets**
  - Made of lead block
  - Kept together by stainless steel frame (target1) , or in aluminum (target2)
  - Alpha emitters come from direct contamination
  - n\_TOF target #1 : 2000-2004
  - n\_TOF target #2 : 2008-2018
- **ISOLDE targets**
  - Different targets materials
  - Potential alpha emitters come from Uranium, Lead and Tantalum targets
  - Targets are irradiated from few days up to couple of weeks

N-TOF



ISOLDE



# Radionuclide inventory

- **Predicting Fluence spectra for the targets – FLUKA simulation**
- **Establishing the full list of radionuclide inventory – using Actiwiz**
  - For most relevant targets
  - For different irradiation times
  - For cooling times ranges from, 6 month – 30 years
- **Reduced radionuclide list – (under certain criteria and assumptions)**
  - Ex: Normalization to hazard factor IRAS = 10.
  - Half-life > 6 months
  - Excluded radionuclides (natural radionuclides): Th-232, Ra-226 and K-40.
- **typical radionuclides of uranium contamination were added – associated with target production**

Radionuclide	ETM/DTM/Hybrid
Mo-93	DTM
Nb-93m	DTM
Tc-99	DTM
Cd-113m	DTM
Sn-119m	DTM
Cs-137	ETM
Ce-144	ETM
Pm-145	ETM
Sm-145	ETM
Pm-147	DTM
Gd-153	ETM
Tb-157	DTM
Lu-174	ETM
Hf-178n	ETM
Po-208	DTM
Po-209	DTM
Pb-210	DTM
Ac-227	DTM
Ra-228	DTM
Th-228	DTM
Pa-231	DTM
U-232	DTM
U-234	DTM
U-235	Hybrid
Pu-236	DTM
Np-237	DTM
Pu-238	DTM
U-238	ETM
Pu-239	DTM



# Gamma spectrometry Analysis

- **Gamma spectrometry to quantify MDA for each radionuclide**
- **Based on the MDA value, a radionuclide can be classified to**
  - ETM –  $MDA < 0.5$  declaration threshold (criteria set by waste repository)
  - Hybrid –  $0.5DT < MDA < DT$
  - DTM –  $MDA > DT$ , or no MDA (not a gamma emitter)
- **Measurement set-up specifications**
  - 42 cm between the drum and the detector,
  - 1800 seconds counting time,
  - Geometry is standardized,
  - Double measurements every 20 drums (random quality controls),

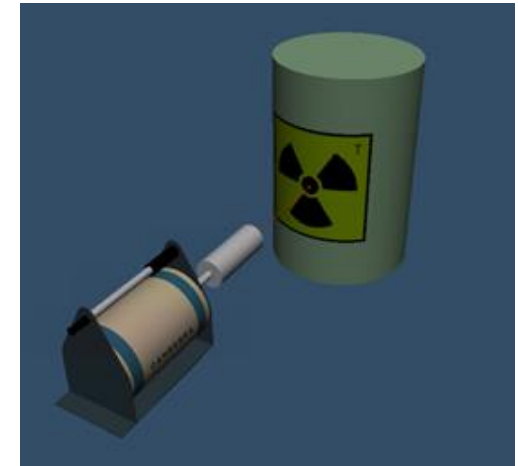
Edit dimensions - Simple Cylindre

Description: cyl\_mde042cm  
Comment:

Unit: mm cm m in ft

No.	Description	d.1	d.2	d.3	d.4	d.5	Material	Density	Rel. Conc.
1	Conteneur	1.25	571.5	855			304ss	7.81	
2	Source - Couche	855					pvc	0.2	1.00
3	Source - Couche	0						0	0.00
4	Absorber 1	0						0	
5	Absorber 2	0						0	
6	Source-Detector	420	0	0	0	0			

Buttons: OK, Cancel, Apply, Help, View Drawing...



# Scaling Factors

- **Analytical scaling factors**
  - Obtained by a ratio between specific activity of KN and DTM/hybrid
- **Experimental Scaling factors**
  - Obtained by radiochemical analysis of samples
  - For BEEP-ABG
    - Experimental SFs were only included for structural material of the target, taken from SHERPA
    - For target core, only analytical scaling factors were considered
- **relying exclusively on Monte Carlo and analytical calculations ( for contamination with alpha)**
- **Agreement between analytical and experimental SFs were shown in previous studies**

KN	6 months to 3 years					
	Ru-106	Bi-207	Hf-172	U-238	Na-22	Co-60
<b>DTM/Hybrid</b>						
H-3	1.8	306.7	1.4		3.3	2.7
Be-10						
C-14					3.9E-03	
Cl-36						
Ar-39						
Ca-41					9.2E-04	
V-49						3.2
Fe-55					17.7	43.2
Ni-63						1.1
Zn-65	3.7E-03	0.4	3.0E-03			
Ge-68	9.1E-04	0.2	9.5E-04			
Kr-85	2.8E-02	0.6				
Sr-90	0.1	0.8				
Mo-93		1.2E-03				
Nb-93m						
Tc-99						
Ag-108m	2.7E-04	4.8E-03	5.3E-06			
Cd-113m						
Sn-119m	0.3					
Sb-125	0.3	0.1				
Pm-147	0.2					
Gd-148	2.7E-04	0.3	1.4E-02			
Eu-152						
Tb-157						
Ta-179	3.3E-02	25	3.1			
Pt-193		10				
Au-195	2.5E-02	80				
Pb-202						
Tl-204		184.1				
Po-208	0.1					
Po-209						
Pb-210	7.1E-03					

# Calculation of hazard factor - IRAS

- **Extracts from gamma-spec**

- Meta data, Detected radionuclides, MDA

- **Apply the criteria**

- **calculate activities**

- Activities of ETMs
- Activities of DTMs/Hybrid

- **Calculate IRAS**

- $IRAS = \sum_i \frac{a_i}{L_i}$ ,
- $a_i$  is the specific activity and  $L_i$  the reference activity limit for the radionuclide  $i$

## Example of different criterion for different RN type

Radionuclide	Radionuclide
ETM	<ul style="list-style-type: none"><li>• activity contributes to IRAS if it is &gt; DT(ANDRA)</li><li>• activity maybe reported activity or MDA</li></ul>
Hybrid	<ul style="list-style-type: none"><li>• detected activity,</li><li>• if not, calculated activity(KN_activity, SF_KN)</li></ul>
DTM	always calculated activity with SFs (analytical or experimental)

# CIRAS

- **To calculate the IRAS factor of a waste package**
  - from the results of the gamma spectrometry reports and
  - from a set of scaling factors,
- **To establish the list of radionuclides and their activities**
- **User friendly**
- **Possibility to add new elimination pathways and modify input data**

The screenshot displays the CIRAS web application interface. At the top, there is a blue header with the CIRAS logo and a user login status: "Welcome to: Mohammad Rababah (logged as mrababah) Log". Below the header is a progress bar with four steps: "Choose an action", "Files", "IRAS results" (the current step, marked with a '3'), and "Export". The main content area shows a report for "HCPWPNG005-T3000113 / 0000000 / 0BA" with a calculated IRAS factor of 0.11 highlighted in green. The report also includes a table of radionuclides and their activities.

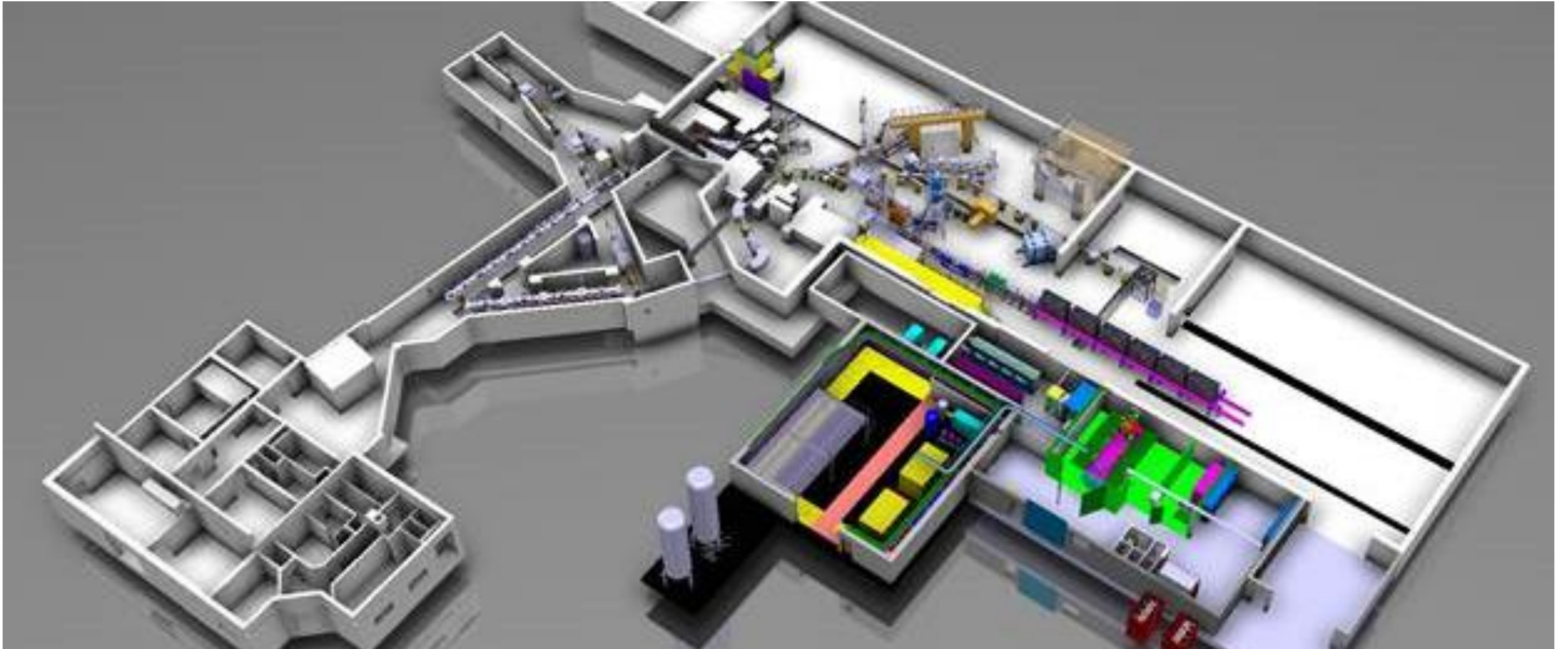
Radionuclide	Activity
H-3	1.32E+1
Na-22	5.28E-3
Co-60	1.81E-2
Ag-108m	7.25E-3
Cs-137	3.88E-2
Hf-178n	2.28E-3
Bi-207	6.69E-2
U-238	1.67E-1

Acknowledgement to Xavier Eric Ouvrard, the software developer.

# Conclusion

- **The methodology comprised several steps**
  - Monte-Carlo Fluka simulation
  - Activiz calculation
  - Gamma-spectrometry analysis
  - Criteria application
  - Establishing final radionuclide inventory, activities, IRAS.
- **This methodology can be extended to other waste produced at similar facilities or research laboratories**

# Questions





[home.cern](http://home.cern)